

50-320

June 23, 1986  
NRC/TMI-86-061

MEMORANDUM FOR: Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Frank J. Miraglia, Director  
Division of PWR Licensing-8

FROM: William D. Travers, Director  
TMI-2 Cleanup Project Directorate

SUBJECT: NRC TMI-2 CLEANUP PROJECT DIRECTORATE WEEKLY STATUS  
REPORT FOR JUNE 16 - JUNE 22, 1986

1. DEFUELING

- Pick and place defueling activities continued during the week but were suspended near the end of the week to clean and strip the work platform in preparation for receiving the drilling rig for core stratification. One additional canister was transferred to the fuel pool bringing the total number of canisters transferred to 43.
- Two additional internals vent valves were removed. Removal of additional vent valves will be scheduled after the core boring.
- On Sunday, June 15, 1986 a small piece of fuel material was handled without proper radiological controls evaluation. A piece of fuel fell onto the defueling platform when removing a defueling tool from the reactor. The radiological controls technician, working on the defueling platform performed an initial survey of the material and obtained a reading of greater than 5 R/hr on contact. Without further survey or direction, the technician placed the particle back in the vessel using thick rubber gloves and a towel. A second piece of material was discovered on June 17, 1986 and returned to the vessel. Proper surveys were performed in this case and the particle was remotely replaced. No additional exposure resulted on June 17. The licensee, at NRC request, took immediate corrective actions to prevent reoccurrence. These actions included performing a dose assessment for the individual involved in the June 15 event, re-instructing all persons working in the reactor building and command centers on small particle hazards, and providing remote tools to handle particles. The preliminary dose assessment for the individual was 12 mrem. The NRC has confirmed that all other immediate corrective actions have been completed. Assigned Health Physics Specialists are conducting a special inspection regarding the event.

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2. PLANT STATUS

- The reactor remains in long term cold shutdown, vented to the atmosphere. Core cooling is by natural heat loss to ambient building atmosphere. The average incore thermocouple reading is 80°F.
- The airborne radioactivity on the defueling platform is about 3 E-7 uCi/cc Tritium and 4 E-10 uCi/cc particulates, predominately Cesium-137 and Strontium-90. The platform is mounted above the modified internals indexing fixture which is mounted over the reactor vessel. These provide about 15 feet of water over the core region and 6 feet over the carousel holding the defueling canisters.

3. WASTE MANAGEMENT

- The Submerged Demineralizer System (SDS) is continuing to process batch S-133. SDS processing to date has been 4,011,603 gallons.
- EPICOR II completed processing batch 290, 291, and 292. The system has processed 2,969,106 gallons to date.

4. ENVIRONMENTAL MONITORING

- US Environmental Protection Agency (EPA) sample analysis results show that TMI site liquid effluents are in accordance with regulatory limits, NRC requirements, and The City of Lancaster Agreement.
- The Lancaster water sample taken at the water works river intake and analyzed by EPA consisted of a seven day composite sample taken June 1 - 7, 1986. A gamma scan detected no reactor related activity.
- TMI water samples taken by EPA at the plant discharge (includes Units 1 and 2) to the river consisted of seven daily composite samples taken from May 31 - June 7, 1986. A gamma scan detected no reactor related activity.
- The EPA analysis of the NRC outdoor air sample for the period June 13 - 19, 1986 showed that concentrations of Cs-137 and I-131 were below the lower limit of quantitative detectability for the system.

5. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- A fuel canister D-138 was aligned, grappled, and transferred from the west upender to a canister storage rack.
- Assembly of the desludging system continued.
- The testing program was completed on the transfer shield, cask upender, and other components needed to support fuel shipping and the system was turned over to plant operations for training.

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6. HRC EVALUATIONS IN PROGRESS

- Technical Specification Change Request number 49, and 51.
- Recovery Operations Plan Change number 31, 33, 35, and 36.
- Solid Waste Facility Technical Evaluation Report.
- Reactor Building Sump Criticality Safety Evaluation Report.
- Defueling Canister Technical Evaluation Report, Revision 2.
- The Core Stratification Sample Acquisition, Revision 4, Safety Evaluation Report (SER) and the Canister Handling and Preparation for Shipment SER were issued on June 19 and June 20, 1986, respectively. The Core Stratification Sample SER includes a provision for leaving a drill casing in the damaged core rubble to allow television camera access to the lower core regions. The Canister Handling and Preparation for Shipment SER discusses fuel canister movement from the dewatering station to the shipping cask.

**ORIGINAL SIGNED BY:**

*William D. Travers*

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4

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